LA-UR-22-31190

Approved for public release; distribution is unlimited.

- Title:
 Processing files with NJOY Reading and manipulating ACE files with ACEtk
- Author(s): Haeck, Wim
- Intended for: 2022 MCNP User Symposium, 2022-10-17/2022-10-21 (Los Alamos, New Mexico, United States)

Issued: 2022-10-21









Los Alamos National Laboratory, an affirmative action/equal opportunity employer, is operated by Triad National Security, LLC for the National Nuclear Security Administration of U.S. Department of Energy under contract 89233218CNA000001. By approving this article, the publisher recognizes that the U.S. Government retains nonexclusive, royalty-free license to publish or reproduce the published form of this contribution, or to allow others to do so, for U.S. Government purposes. Los Alamos National Laboratory requests that the publisher identify this article as work performed under the auspices of the U.S. Department of Energy. Los Alamos National Laboratory strongly supports academic freedom and a researcher's right to publish; as an institution, however, the Laboratory does not endorse the viewpoint of a publication or guarantee its technical correctness.



Processing files with NJOY Reading and manipulating ACE files with ACEtk

W. Haeck

2022 MCNP User Symposium, October 17-21, 2022



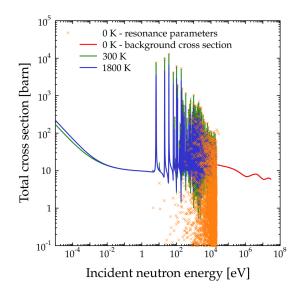
Outline

- So you have the evaluated data, now what?
- NJOY processing for MCNP libraries
- Overview of the ACE format and structure
- The ACEtk toolkit



So, you have the evaluated data - now what?

- What we have: evaluated nuclear data libraries
 - Not necessarily in a "simple" format and processing might be needed before use
 - Incident neutron and charged particle data
 - Covariance data
 - Fission yield data, radioactive decay data
 - Photoatomic and photonuclear data
 - Thermal scattering data
- What we need: nuclear data application libraries
 - A subset of the data for the particular application
 - Provides derived data not available in evaluated data:
 - Temperature dependent data
 - Energy deposition data, etc.





So, you have the evaluated data - now what?

- This is where the XCP-5 Nuclear Data Team at LANL comes into play:
 - Produce and maintain nuclear data libraries for LANL simulation codes
 - Verify and validate new data libraries when they become available
- NJOY is the nuclear data processing software developed at Los Alamos
 - Initially developed in the '70s as a single package to replace individual programs
 - Originally written in Fortran-77
 - Known as MINX-II prior to a printer malfunction

$$M + 1 = N$$

 $I + 1 = J$
 $N + 1 = O$
 $X + 1 = Y$

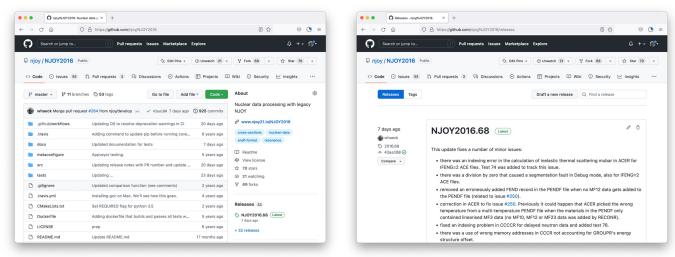


Which version of NJOY should you use?

- NJOY has been around for over 40 years now
 - Major versions: NJOY99, NJOY2012, NJOY2016, NJOY21
- NJOY2016 is the production version in use at LANL
 - The MCNP ENDF/B-VIII.0 library was produced using NJOY2016
 - Latest version is NJOY2016.68 (September 2022)
- NJOY21 is in essence a NJOY2016 wrapper
 - It provides additional input verification
 - Latest version is NJOY21 v1.2.2 (January 2021)
 - We advice you to use NJOY2016 instead



Get it at https://github.com/njoy/NJOY2016



- Latest version is NJOY2016.68 (September 2022)
 - We aim to release updates every three months even if the changes are minor
 - This coincides with quarterly reports that we give to our funding sources

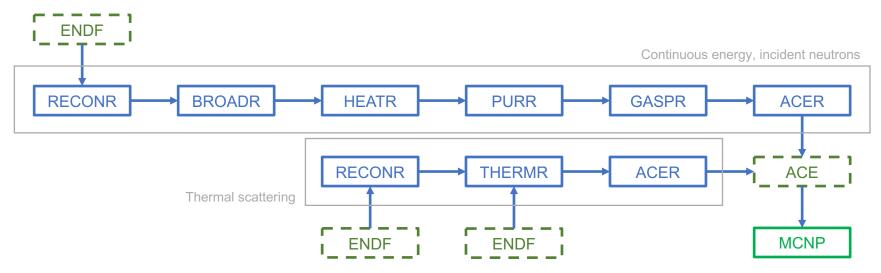


- Prerequisites:
 - git
 - cmake 3.15 or higher
 - a Fortran 2003 compliant compiler such as gcc-7 or higher
- Installation instructions:

```
git clone https://github.com/njoy/NJOY2016.git
cd NJOY2016
mkdir build
cd build
cmake -DCMAKE_BUILD_TYPE=Release ../
make -j8
```



- NJOY provides a set of data processing modules that are called sequentially
 - Different processing paths for different library types and applications
 - Incident neutron, incident charged particles, thermal scattering, photonuclear, etc.

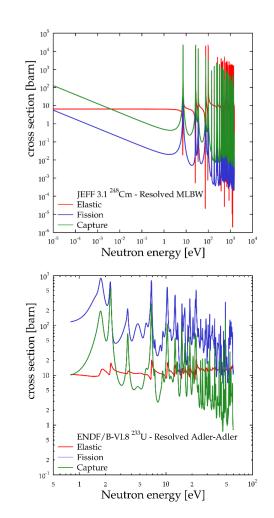




Module example: RECONR

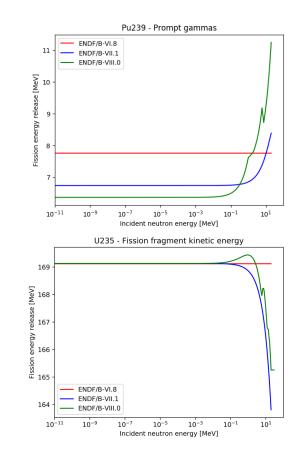
- Reconstruction and linearisation of cross sections
 - Takes the resonances parameters and computes cross sections for total, elastic, fission and capture
 - Takes the other cross sections and linearises them
 - Puts all reactions on the same energy grid
- Important input parameters
 - Fractional reconstruction tolerances
 - Maximum integral error
- Input: reconr

 20 21
 `A fancy title for the new tape'
 9228 0 0
 0.001 /
 0



Other notable NJOY modules

- BROADR: temperature dependent cross sections
- GROUPR: multi-group cross sections
- HEATR: calculate KERMA and DPA cross sections
- THERMR: thermal scattering data
- GASPR: charged particle production cross sections
- PURR: unresolved resonance probability tables
- ACER: produce ACE libraries for MCNP
- PLOTR & VIEWR: visualisation of nuclear data
- And many more ...





moder 20 - 25reconr -25 -21 'AM241 - 293.6 K - ENDF/B-VIII.0 (NJOY2016.68) '/ 9543 0 0 0.001 0 0.01 5e-08 0 / broadr -25 -21 -22 9543 1 0 0 0 0.001 le+06 0.01 5e-08 293.6 0 / heatr -25 -22 -21 / 9543 5 0 0 0 0 / 302 318 402 442 444 / thermr 0 -21 -22 / 0 9543 16 1 1 0 1 221 2 / 293.6 0.001 5.0 gaspr -25 -22 -21 /

unresr -25 -21 -22 9543 1 9 1 293.6 1e+10 1e+8 1e+6 1e+4 1e+3 3e+2 1e+2 3e+1 1e+1 0 / purr -25 -22 -21 9543 1 9 20 64 1 0 293.6 1e+10 1e+8 1e+6 1e+4 1e+3 3e+2 1e+2 3e+1 1e+1 0 / acer -25 -21 0 40 41 1 0 1 .02 / 'AM241 - 293.6 K - ENDF/B-VIII.0 (NJOY2016.68)'/ 9543 293.6 1 1 1 acer 0 40 42 40 41 7 1 1 -1 / 'AM241 - 293.6 K - ENDF/B-VIII.0 (NJOY2016.68)'/ stop



NJOY output is worth looking at ...

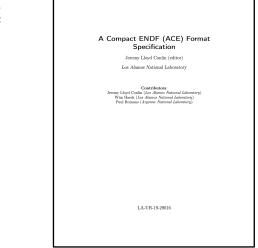
- · Something is wrong with this output for elemental sulphur
 - Can you guess what it is?
 - The original evaluation dates back to 1979, as old as I am

estimated maximum error due to resonance integral check (errmax, errint)

	-1					
upper	elastic	percent	capture	percent	fission	percent
energy	integral	error	integral	error	integral	error
1.00E-05						
1.00E-04	2.26E+00	0.000	3.58E+01	0.000	1.24E-01	0.000
1.00E-03	2.26E+00	0.000	1.13E+01	0.000	3.92E-02	0.000
1.00E-02	2.26E+00	0.000	3.58E+00	0.000	1.24E-02	0.000
1.00E-01	2.26E+00	0.000	1.13E+00	0.000	3.92E-03	0.000
•••						
1.00E+05	1.10E+00	0.003	1.53E-03	0.038	2.01E-03	1.743
2.00E+05	5.86E+00	0.001	2.04E-03	0.247	1.30E-03	1.077
5.00E+05	2.48E+00	0.010	1.22E-03	0.866	6.59E-04	1.765
1.00E+06	1.52E+00	0.018	5.29E-04	1.057	2.69E-05	2.025



- The nuclear data application library files for MCNP are referred to as ACE files
 - Each ACE file contains one or more ACE tables
 - Specifications: https://github.com/NuclearData/ACEFormat
- There are multiple ACE table types:
 - Incident neutron and charged particle ACE tables
 - Photonuclear ACE tables
 - Thermal scattering ACE tables
 - Photoatomic ACE tables
 - Dosimetry ACE tables
 - Multigroup ACE tables



• Each ACE table type has its own structure but some pieces are shared



• Each ACE table has 5 basic components: a header and 4 arrays

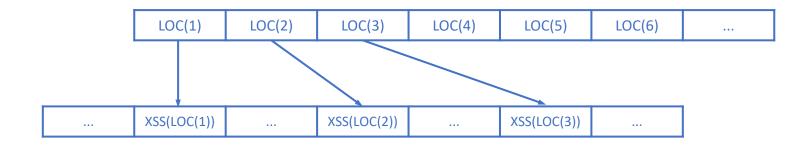
header	92235.00c	233.0248	00 2.530	1E-08 0	5/01/18			
neader	U235 Lib80	x (jlconl	in) Refe	rence LA-	UR-18-240	34 by Con	lin, J.L.	, et al. mat9228
izaw array	0	0.	0	Ο.	0	Ο.	0	0.
	0	0.	0	Ο.	0	Ο.	0	0.
	0	0.	0	Ο.	0	Ο.	0	0.
	0	Ο.	0	Ο.	0	Ο.	0	0.
nxs array	7168374	92235	76027	91	44	583	5	6
	0	92	235	0	0	0	0	0
jxs array	1	380136	380483	380574	380665	380756	380847	4018427
	4018472	4157159	4157203	5083731	5159758	5160341	5160924	5166428
	5167011	5167011	5167594	5351569	456964	5351650	5070901	5072750
	5072763	5072805	5072811	0	0	5351651	5351656	5351661
	1.00000	000000E-1	1 1.031	2500000E	-11 1.0	625000000	0E-11 1	.09375000000E-11
xss array	1.12500	000000E-1	1 1.156	25000000E	-11 1.1	875000000	0E-11 1	.21875000000E-11
	1.25000	000000E-1	1 1.281	2500000E	-11 1.3	125000000	0E-11 1	.34375000000E-11
	• • •							



- The header information (multiple header types are available)
 - The most important information: the ZAID, the atomic weight ratio and the temperature
- The izaw array: 16 pairs of ZA and atomic weight ratio values
 - Essentially only used in thermal scattering files
- The nxs array: 16 integers with table related information
 - Things like number of reactions, number of secondary particle types, etc. go here
- The jxs array: 32 integers that function as locators to specific ACE blocks
 - Locators are <u>always 1-based</u> indices into the xss array
- The xss array: a single array of real values containing blocks of data
- The interpretation of the <code>nxs</code>, <code>jxs</code> and <code>xss</code> array differs by ACE table type



- The xss array is a flat array of real values interpreted through locators
 - The jxs array contains the locators for an ACE table's main data blocks
 - The xss array can contain locators to secondary data blocks
 - All locators are <u>1-based absolute or relative indices</u> because Fortran
 - Locators only point to the beginning of a data block (there can be "gaps")





- Let's take a look at the incident neutron and charged particle ACE tables
- The xss array can be subdivided into 3 main pieces
 - Primary particle data
 - Everything MCNP needs to transport the primary particle
 - Additional data such as heating data
 - Distribution data for outgoing photons
 - Only used when transporting photons
 - Distribution data for other secondary particle types
 - Only used when transporting those particle types

Data for the primary particle
Optional data for outgoing photons
Optional data for other secondary particle types



Principal cross section block (ESZ) Optional fission multiplicity block (NU) Reaction number block (MTR) Data for the primary particle Reaction Q-value block (LQR) Frame and multiplicity block (TYR) Cross section block (LSIG and SIG) Optional data for outgoing Angular distribution block (LAND and AND) photons Energy distribution block (LDLW and DLW) Optional data for other Optional unresolved ptable block (UNR) secondary particle types Optional delayed neutron data blocks (DNU, BDD and DEND)

These last 2 blocks often appear after the photon data

Principal cross section block (ESZ)

Optional fission multiplicity block (NU)

Reaction number block (MTR)

Reaction Q-value block (LQR)

Frame and multiplicity block (TYR)

Cross section block (LSIG and SIG)

Angular distribution block (LAND and AND)

Energy distribution block (LDLW and DLW)

Optional unresolved ptable block (UNR)

Optional delayed neutron data blocks (DNU, BDD and DEND)

These last 2 blocks often appear after the photon data

Common energy grid

Total cross section values

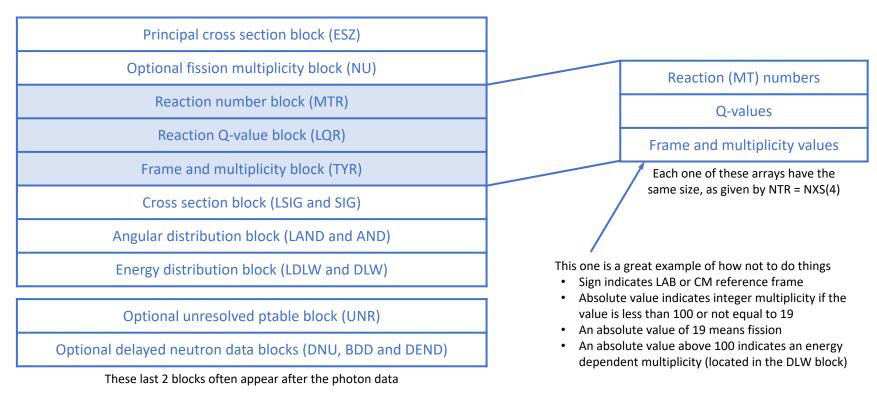
Elastic cross section values

Disappearance cross section values

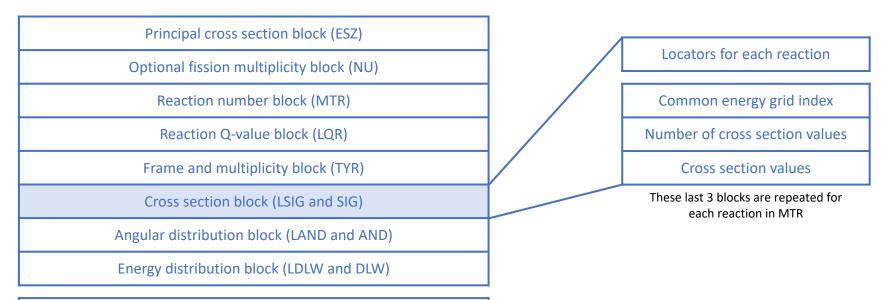
Heating numbers

Each one of these arrays have the same size, as given by NES = NXS(3)







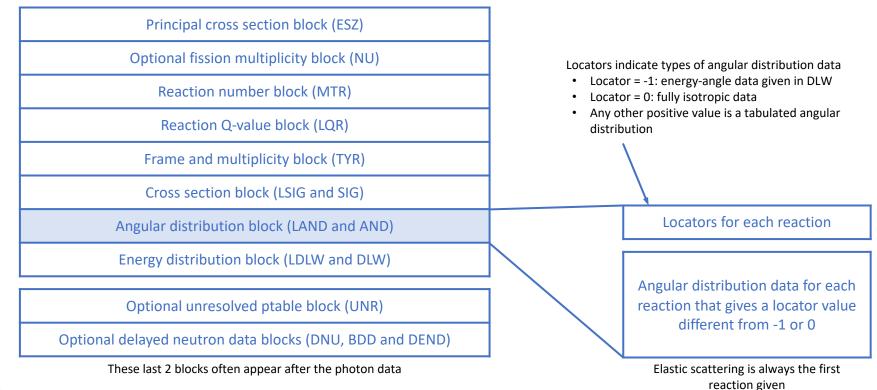


Optional unresolved ptable block (UNR)

Optional delayed neutron data blocks (DNU, BDD and DEND)

These last 2 blocks often appear after the photon data





10/18/22

Principal cross section block (ESZ) Optional fission multiplicity block (NU) Reaction number block (MTR) Data for the primary particle Reaction Q-value block (LQR) Frame and multiplicity block (TYR) Cross section block (LSIG and SIG) Optional data for outgoing Angular distribution block (LAND and AND) photons Energy distribution block (LDLW and DLW) Optional data for other Optional unresolved ptable block (UNR) secondary particle types Optional delayed neutron data blocks (DNU, BDD and DEND)

 \bigotimes

These last 2 blocks often appear after the photon data

Photon production block (GPD)

Photon production reaction number block (MTRP)

Photon production cross section block (LSIGP and SIGP)

Photon angular distribution block (LANDP and ANDP)

Photon energy distribution block (LDLWP and DLWP)

Photon multiplicity reaction number block (YP)



Optional data for outgoing photons

Optional data for other secondary particle types



Secondary particle information block (PTYPE)

Secondary particle locator block (IXS)

Secondary particle production block (HPD)

Secondary particle reaction number block (MTRH)

Secondary particle frame block (TYRH)

Secondary particle cross section block (LSIGH and SIGH)

Secondary particle distribution block (LANDP and ANDP)

Secondary particle energy distribution block (LDLWH and DLWH)

Secondary particle multiplicity reaction number block (YH)

These last 7 blocks are repeated for each secondary particle type in PTYPE

Data for the primary particle
Optional data for outgoing photons
Optional data for other secondary particle types

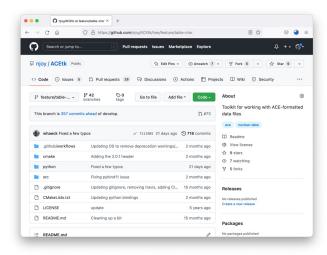


- And this is only the top of the iceberg
- The ACE format has certain idiosyncrasies
 - Locator logic is not consistent
 - Storing multiple pieces of data in a single field (e.g. TYR)
 - Subtle differences depending on where a block appears (e.g. SIG, SIGP, SIGH)
 - And many more ...
- Solution: do not interact with the ACE file directly, use an interface instead
 - ACEtk: this interface abstracts away some of the ACE idiosyncrasies



The ACEtk toolkit

- ACEtk: https://github.com/njoy/ACEtk
 - A format component developed in the NJOY modernisation project
 - Reading, writing and manipulate ACE files
 - Using a C++ and Python API at the same time
- ACEtk support for the following ACE file types:
 - Incident neutron and charged particle ACE files
 - Photoatomic and photonuclear ACE files
 - Thermal scattering ACE files





The ACEtk toolkit

- Prerequisites:
 - git
 - cmake 3.15 or higher
 - a C++-17 compliant compiler such as gcc-7 or higher
 - Python 3.5 or higher
- Installation instructions:

git clone https://github.com/njoy/ACEtk
cd ACEtk
git checkout feature/table-ctor
mkdir build
cd build
cmake -DCMAKE_BUILD_TYPE=Release ../
make ACEtk.python -j8

